

## UNITED STATES NUCLEAR REGULATORY COMMISSION

## REGION II SAM NUNN ATLANTA FEDERAL CENTER 61 FORSYTH STREET SW SUITE 23T85 ATLANTA, GEORGIA 30303-8931

October 3, 2002

Duke Energy Corporation ATTN: Mr. G. R. Peterson Site Vice President Catawba Site 4800 Concord Road York, SC 29745-9635

SUBJECT: SAFETY SYSTEM DESIGN AND PERFORMANCE CAPABILITY INSPECTION

NRC INSPECTION REPORT NOS. 50-413/2002-08 and 50-414/2002-08

Dear Mr. Peterson:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a safety system design and performance capability inspection at your Catawba facility during January 2003. A team of five inspectors will perform the inspection. The inspection team will be led by Mr. J. Lenahan, a senior reactor inspector from the NRC Region II Office. The inspection will be conducted in accordance with baseline Inspection Procedure 71111.21, Safety System Design and Performance Capability.

The inspection objective will be to evaluate the capability of the component cooling water (KC) system and support systems, as well as other related systems, to perform their design functions. The team will also review scenarios leading to partial or total loss of the KC system.

During a telephone conversation on September 30, 2002, Mr. J. Lenahan of my staff, and Mr. G. Strickland of your staff, confirmed arrangements for an information gathering site visit and the two-week onsite inspection. The schedule is as follows:

- Information gathering visit: November 25 26, 2002
- Onsite inspection: January 6 10 and 27 31, 2003

The purpose of the information gathering visit is to obtain information and documentation outlined in the enclosure needed to support the inspection. Mr. R. Bernhard, a Region II Senior Reactor Analyst, will accompany Mr. Lenahan during the information gathering visit to review PRA data and identify risk significant components which will be examined during the inspection. Please contact Mr. Lenahan prior to preparing copies of the materials listed in the Enclosure. The inspectors will try to minimize your administrative burden by specifically identifying only those documents required for inspection preparation.

During the information gathering visit, the team leader will also discuss the following inspection support administrative details: office space; specific documents requested to be made available to the team in their office space; arrangements for reactor site access; and the availability of

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knowledgeable plant engineering and licensing organization personnel to serve as points of contact during the inspection.

Thank you for your cooperation in this matter. If you have any questions regarding the information requested or the inspection, please contact me at (404) 562-4605, or Mr. Lenahan at (404) 562-4625.

Sincerely,

/RA: ORIGINAL SIGNED BY J. MOORMAN FOR:/

Charles R. Ogle, Chief Engineering Branch 1 Division of Reactor Safety

Docket Nos. 50-413, 50-414 License Nos. NPF-35, NPF-52

Enclosure: Information Request for the Component Cooling Water System

cc w/encl: G. D. Gilbert Regulatory Compliance Manager Duke Energy Corporation Electronic Mail Distribution

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(cc w/encl cont'd - See page 3)

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(cc w/encl cont'd)
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Distribution w/encl: See page 4

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<u>Distribution w/encl</u>: C. Patel, NRR C. Evans (Part 72 Only) RIDSNRRDIPMLIPB PUBLIC

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SIGNATURE	MOORMAN		MOORMAN		CARROLL									
NAME	LENAHAN		MOORMAN		HAAG									
DATE	10/2/2002		10/2/2002		10/3/2002		10/	/2002	10/	/2002	10/	/2002	10/	/2002
E-MAIL COPY?	YES	NO	YES	NO	YES	NO	YES	NO	YES	NO	YES	NO	YES	NO
PUBLIC DOCUMENT	YES	NO				•		•						•

OFFICIAL RECORD COPY DOCUMENT NAME: C:\ORPCheckout\FileNET\ML022800669.wpd

## INFORMATION REQUEST FOR THE SAFETY SYSTEM DESIGN AND PERFORMANCE CAPABILITY INSPECTION:

## **COMPONENT COOLING WATER SYSTEM**

Note: Electronic media is preferred if readily available (i.e., on computer disc).

- Site specific administrative procedures related to standard operation, abnormal operation, and emergency operation of the component cooling water (KC) system, including support systems, and other related systems. Other related systems include, but may not be limited to the essential auxiliary power system, the nuclear service water system, and the demineralized water system.
- Design criteria (i.e., design basis documents) for the KC system and other related systems.
- KC system Technical Specification requirements and a list of associated surveillance test/calibration procedures for the KC system and related systems.
- Copies of applicable sections of the UFSAR for the KC system, and other related systems and copies of applicable sections of changes to the UFSAR which have yet to be docketed.
- KC system, and other related systems piping and instrumentation drawings, one-line diagrams; electrical schematics, and wiring and logic diagrams.
- A list of engineering calculations (Electrical, Instrumentation and Controls and Mechanical/Nuclear) applicable to the KC system, and other related systems.
- A list of plant modifications to the KC system, and other related systems, implemented since 1992.
- List of current open temporary modifications and operator work arounds involving operation of the KC and the other related systems.
- List of Problem Investigation Process Reports (PIPs) initiated since 1992 affecting the KC system, and other related systems.
- Summary of corrective maintenance activities, including the maintenance rule event log, performed on the KC system and other related systems in the past 12 months.
- An index of drawings for the KC system, and other related systems.
- Self-assessment performed on KC system and other related systems in the last 24 months.
- System description and operator training modules for the KC system and other related systems.

- List of Operating Experience Program evaluations of industry, vendor, or NRC generic issues related to the KC system for the past 3 years.
- List of instrument setpoint changes affecting the KC system and related systems initiated since 1992. Include the number and title, date, brief description, and corresponding calculation number.
- PRA Fault Tree Data for the KC.
- PRA/Risk Achievement Worth (RAW) listing for the KC system, and related support systems, evaluated for failure of the KC system.
- A list of PRA system dependencies and success criteria for KC and its support systems.